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AUTHOR(S): E.D. Cashwell, X-6
R.G. Schrandt, X-6

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FLUX AT A POINT IN MCNP

E. D. Cashwell and R. G. Schrandt
Group X-6
Monte Carlo, Applications and Transport Data Group
Theoretical Applications Division
Los Alamos Scientific Laboratory
Los Alamos, New Mexico

ABSTRACT

The current state of the art of calculating flux at a point with MCNP is discussed. Various techniques are touched upon, but the main emphasis is on the fast improved version of the once-more-collided flux estimator, which has been modified to treat neutrons thermalized by the free gas model. The method is tested on several problems of interest and the results are presented.

INTRODUCTION

The next-event estimator (NEE) used in a normal Monte Carlo game for the flux at a detector embedded in a scattering medium suffers from a $(1/r^2)$ -singularity. Consequently, the variance of the estimator is infinite even though the mean is finite.

In 1977, Kalli and Cashwell¹ proposed and evaluated three estimation schemes for flux at a point. A new, once-more-collided flux estimator (OMCFE) was proposed, which differed from those proposed by Kalos in his original paper.² The scheme has a $(1/r)$ -singularity, leading to finite variance and $(1/\sqrt{N})$ -convergence. It is based on a very simple p.d.f. of the path lengths in the sampling of the intermediate collision points. In addition, this simple p.d.f. for the path length was used in two schemes with bounded estimators similar to those proposed by Steinberg and Kalos³ and by Steinberg.⁴ The three schemes were evaluated in a realistic problem using the continuous energy Los Alamos Monte Carlo code MCNG, the forerunner of MCNP.⁵

Once-More Collided Flux Estimator (OMCFE)

In the present discussion we wish to focus on the OMCFE referred to above. This scheme has been incorporated into MCNP and, although some work still remains to be done, we wish to discuss this method in conjunction with other techniques available in MCNP.

The details of the OMCFE as it exists in MCNP are, for the most part, given in Ref. (1). Without repeating the treatment given there, we wish to touch on the main points of the method, as well as mention generalizations of the method to a wider class of problems. The OMCFE is superimposed on the particle history without affecting it. At each collision (or source point), a nonanalog game is played whereby a next collision point A is chosen, from which a contribution to the detector is made. That is, from every real collision point of the particle history, a once-more-collided contribution is made to the detector.

The two main features in determining the intermediate point A of the once-more-collided scheme are:

1. A directional reselection procedure based on the reselection technique of Steinberg and Kalos;³ and
2. A nonanalog p.d.f. $p^*(s)$ which was used by Kalli⁶ in 1972.

In Fig. 1, consider a collision at S with the resulting scattered direction $\bar{\Omega}_0$ in the cone described. Suppose that a new direction $\bar{\Omega}_1$ is chosen by sampling a new angle β_1 uniformly in $(0, \beta_m)$ and a ϕ_1 uniformly in $(0, 2\pi)$. The result is a concentration of scattered directions closer to the line from S to the detector D than would normally occur. Of course, an adjustment factor must be applied to the weight of the particle due to the reselection.

Once the direction $\bar{\Omega}_1$ is chosen, suppose the intermediate point A is selected along this direction from the p.d.f. $p^*(s)$, where

$$p^*(s) = \frac{b}{(\pi/2 - \alpha_1)r^2} \quad (\text{Cf. Fig. 2}). \quad (1)$$

This density function corresponds to α being chosen uniformly in $(\alpha_1, \pi/2)$. Use of $p^*(s)$ leads to another weight adjustment $p(s)/p^*(s)$, where $p(s)$ is the analog p.d.f. for sampling distance to collision.

In the normal OMCFE, the point A is not a real collision point of the particle history. When these calculations involve reselection of direction and the distance to A using $p^*(S)$, as well as the normal next-event estimator, they tend to be time-consuming. In order to speed up calculations using the OMCFE:

1. Draw an imaginary sphere around the detector;
2. If the collision point S_1 is outside the sphere but the direction after the collision is within the cone defined by S_1 and the sphere, calculate the once-more-collided flux contribution by performing the directional reselection in the cone and calculate the intermediate point A by using $p^*(s)$;

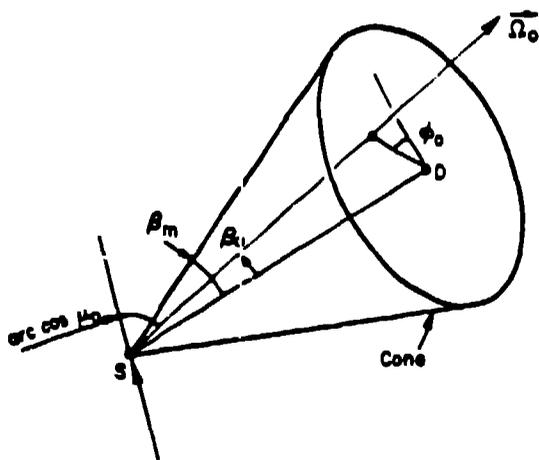


Fig. 1. Geometry in the reselection of a direction.

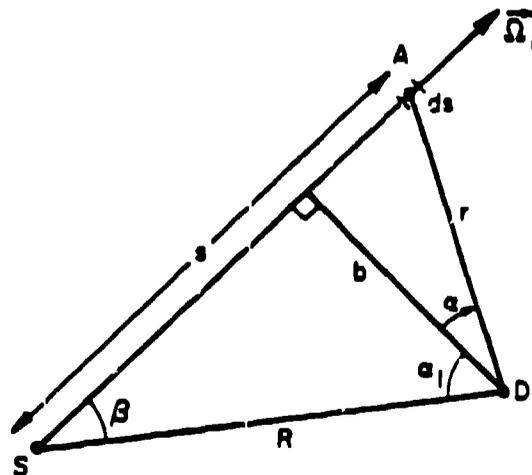


Fig. 2. Geometry in the selection of the intermediate collision point A.

3. If the collision point S_1 is in the sphere and the direction $\vec{\Omega}_0$ after collision is in a 2π -cone (i.e., $\beta_0 < \pi/2$) about the line from S_1 to D, the once-more-collided point is calculated by reselection of $\vec{\Omega}_1$ and using $p^*(s)$ to determine the intermediate point A; if the direction $\vec{\Omega}_0$ after collision is such that $\beta_0 > \pi/2$, no reselection is performed but the intermediate point A is chosen from $p^*(s)$; and

4. Otherwise, calculate the normal next-event contribution from the following collision point S_{1+1} .

The recipe as outlined above works very well in most problems containing ordinary materials. However, in non-thermal problems containing H, the forward scattering off H in the laboratory system of coordinates lead to some modification of the recipe because of the directional reselection procedure. Furthermore, the random motion of the target atoms combined with the motion of the neutron in the thermal routine using the free gas model in MCNP leads to rather extensive modifications for the same reason. The imaginary sphere around the detector may have to be reduced in size in the course of the calculation, as a result of using the reselection procedure.

With the necessary modifications, MCNP is able to treat problems of the types mentioned above, as illustrated by the sample calculations below. Several considerations led to the implementation of the OMCFE rather than one of the schemes leading to a bounded estimator in Ref. (1). First of all, the OMCFE was judged to be the simplest to insert into MCNP. Furthermore, the estimation of flux simultaneously at several points causes no problems in the OMCFE scheme. Finally, since the OMCFE does not alter the particle histories, its use has no effect on other tallies which may be required in a particular problem.

DXTRAN

Let us describe briefly a subroutine, DXTRAN, which has been used in Los Alamos for some years and is an option available in MCNP.⁵ We shall indicate its usefulness in our examples below. DXTRAN is of value in sampling regions of a problem which may be insufficiently visited by particle histories to yield adequate statistical accuracy in a given tally. To explain how the scheme works, let us consider the neighborhood of interest to be a spherical region surrounding a designated point P_0 in space. In fact, we consider two spheres of arbitrary radii about the point $P_0(x_0, y_0, z_0)$. We assume that the particle having direction (u, v, w) collides at the point (x, y, z) , as shown in Fig. 3. The quantities L , θ_I , θ_O , η_I , η_O , R_I , and R_O are clear from the figure. Let us somehow choose a point P_S on the outer sphere and assume that a scattered particle (let us call it a "pseudo-particle" for the moment) is placed there. We give this pseudo-particle a weight equal to the weight of the incoming particle at P_1 multiplied by the ratio of the p.d.f. for scattering from P_1 to P_S with no collision to the p.d.f. for choosing the point P_S in the first place.

If we sample directions isotropically in the cone defined by P_1 and the outer sphere, the number of directions falling inside the inner cone and the number falling in the outer cone will be proportional to $1-\eta_I$ and $\eta_I-\eta_O$, respectively. Let Q be a factor which measures the weight or importance which one assigns to scattering in the inner cone relative to scattering in the outer cone. We now proceed by the following steps:

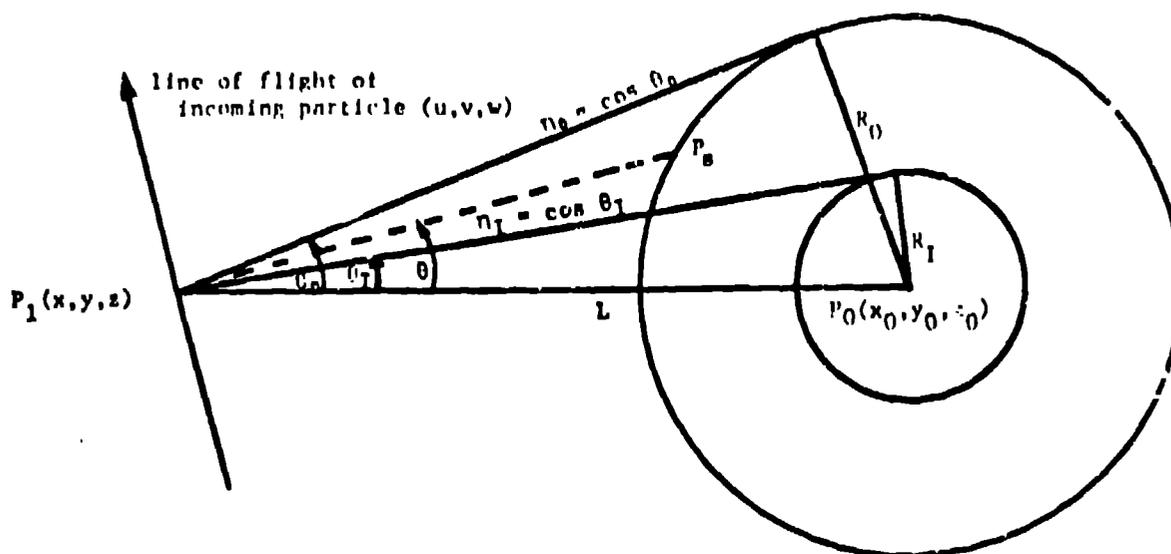


Fig. 3. The geometry of DXTRAN.

1. Sample η uniformly in $(\eta_I, 1)$ with probability $Q(1-\eta_I)/[Q(1-\eta_I) + \eta_I - \eta_0]$; and with probability $(\eta_I - \eta_0)/[Q(1-\eta_I) + \eta_I - \eta_0]$ sample η uniformly in (η_0, η_I) ;

2. Having chosen θ from $\eta = \cos \theta$, we use the scattering formulas in the code to scatter through an angle θ (and an azimuthal angle ϕ chosen uniformly in $(0, 2\pi)$) from the initial direction $\left(\frac{x_0 - x}{L}, \frac{y_0 - y}{L}, \frac{z_0 - z}{L}\right)$, determining a new direction (u', v', w') . Advance the pseudo-particle in the direction (u', v', w') to the point P_2 on the surface of the outer sphere. The new coordinates are saved;

3. The weight attached to the pseudo-particle is the weight of the particle at collision multiplied by

$$v \cdot \frac{P(\eta) [Q(1 - \eta_I) + \eta_I - \eta_0] \exp\left\{-\int_{P_1}^{P_2} \Sigma_t(\epsilon) d\epsilon\right\}}{Q}, \quad \eta_I < \eta < 1$$

and

$$v \cdot P(\eta) [Q(1 - \eta_I) + \eta_I - \eta_0] \exp\left\{-\int_{P_1}^{P_2} \Sigma_t(\epsilon) d\epsilon\right\}, \quad \eta_0 < \eta < \eta_I$$

where

$$\mu = uu' + vv' + ww''$$

$P(\mu)$ = p.d.f. for scattering through the angle $\cos^{-1}\mu$ in the lab system for the event sampled at (x, y, z) .

v = number of neutrons emitted from the event.

Since a collision supplies a particle (let us now drop the term pseudo-particle - these particles are as real as any others) to the outer DXTRAN sphere, the particles from the collision at P_1 are picked up and followed further, but they are killed if they attempt to enter the sphere. It is apparent from the discussion above that this routine has certain features in common with a point detector routine.

This routine is used in a couple of the problems discussed below. In one problem, it is used to obtain the average flux in a small volume as a check against the result obtained from the OMCPE. In another, it is used to help get particles in the vicinity of a detector. While DXTRAN can be useful in many problems, it must be pointed out that the method is time-consuming, being similar in nature to a point detector routine. Further, attention must be paid to the problem of obtaining a sufficient number of histories in the vicinity of the DXTRAN sphere, not just inside the sphere.

CALCULATIONS

The problems discussed below were chosen to demonstrate the behavior of the OMCFE in a variety of settings, with some emphasis on the treatment of H and, in particular, its behavior in the presence of neutrons thermalized according to the free gas model. Illustrations of how DXTRAN can be useful, either as an aid to the OMCFE or as an aid in computing the average flux in a region with a track-length estimator, occur in two of the problems.

The geometries displayed in our problems are deliberately kept simple, partly so that we can display the so-called "exact flux", which is calculated very accurately (to a fraction of a percent) using a surface crossing estimator in the spherical geometry. In the schematics showing the geometry used, not every surface appears. Frequently, additional surfaces were added for the purposes of splitting and Russian roulette, or for the purpose of obtaining average flux in a region, but few surfaces were added in any one calculation.

In each problem, the source at the center of the sphere was chosen to be monoenergetic and isotropic in direction. As easily anticipated, it was found useful to use an exponential biasing to direct more particles toward the detectors. The latter were always placed on a radius of the sphere - say the positive x-axis. The initial flight of a neutron was chosen by sampling μ , the cosine of the angle the starting direction makes with the x-axis, from a p.d.f. $\sim e^{k\mu}$, with k a fixed parameter. The value of k used in each problem is listed on the schematic for that problem.

A feature of MCNP which was used in these calculations has to do with contributions to the detector D from collisions several free paths from the detector. E.g., when collisions occur more than x free paths from D, by playing Russian roulette one can permit, say, only one in ten collisions on the average to contribute to D, with weight enhanced by a factor of ten. The number x is set by the user and in these calculations was usually set to four. This feature of the code can save appreciable amounts of machine time in large systems.

Other information on the schematic which is of interest include the number density of atoms in the material used; the thermal temperature of the problem (if any); the average m.f.p. $\bar{\lambda}$, computed by MCNP over the course of the problem; the source energy and energy cut-off (if any); the time on the CDC-7600 for a given sample of starting neutrons; and the imaginary sphere radii used in the OMCFE and in DXTRAN.

Figs. 4-12 display the geometries and graphs of the results for four problems. Table 1 gives a comparison of the final flux values at the end of each run with the "exact values". The errors in the final fluxes also appear.

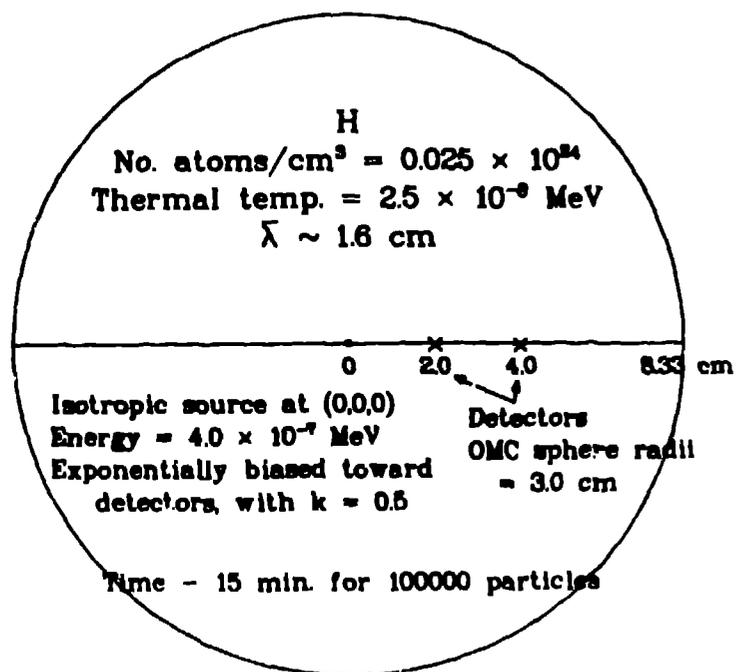


Fig. 4 Geometry for Thermal Hydrogen Problem.

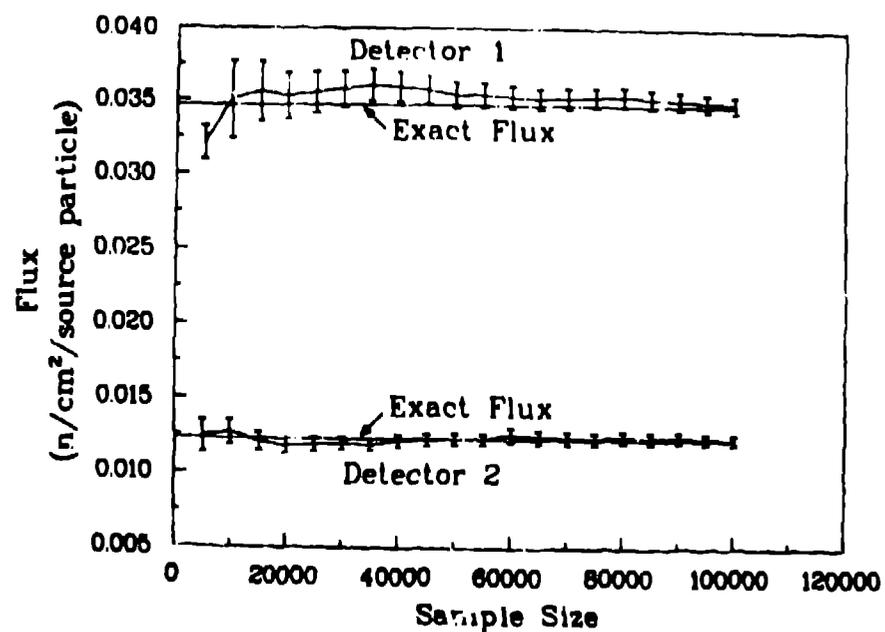


Fig. 5 Flux at Two Detectors in Thermal H with the OMCFE.

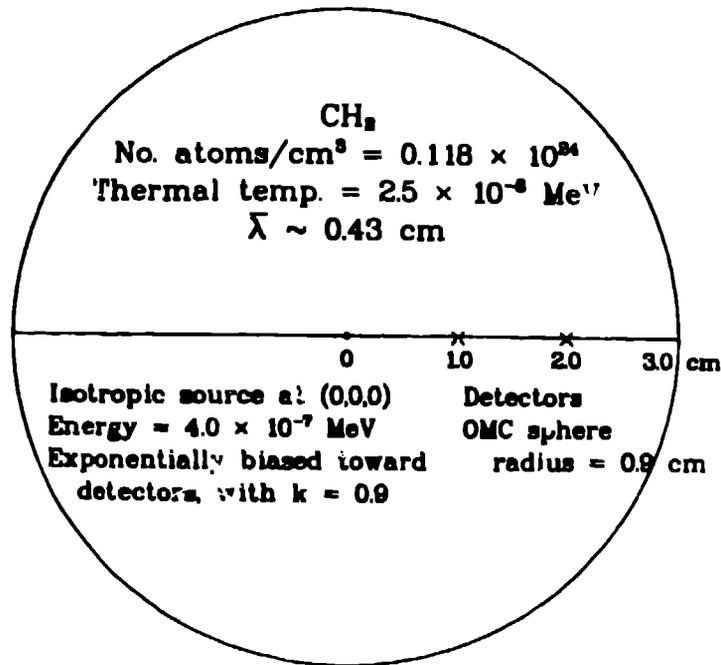


Fig. 6 Geometry for Thermal CH₂ Problem.

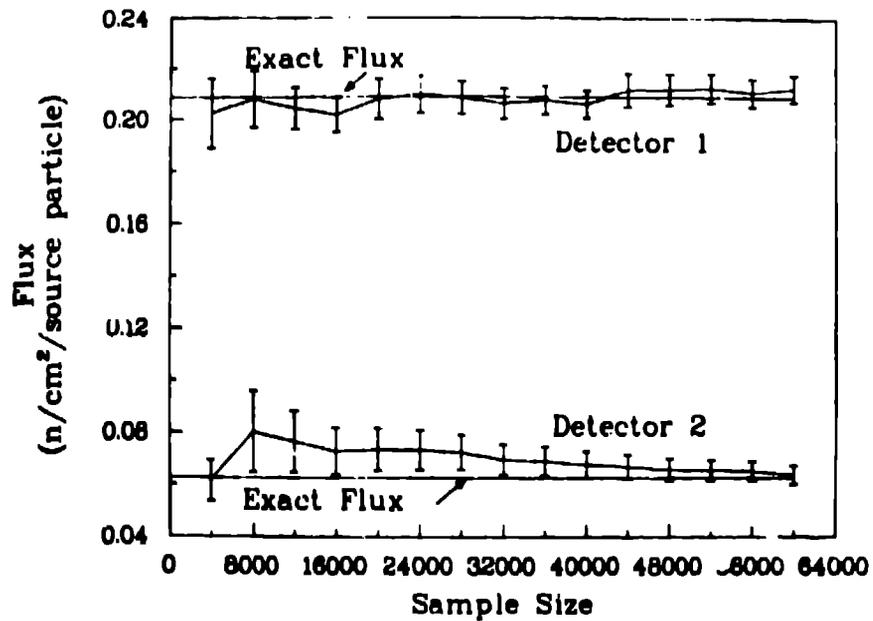


Fig. 7 Flux at Two Detectors in Thermal CH₂ with the OMCFE.

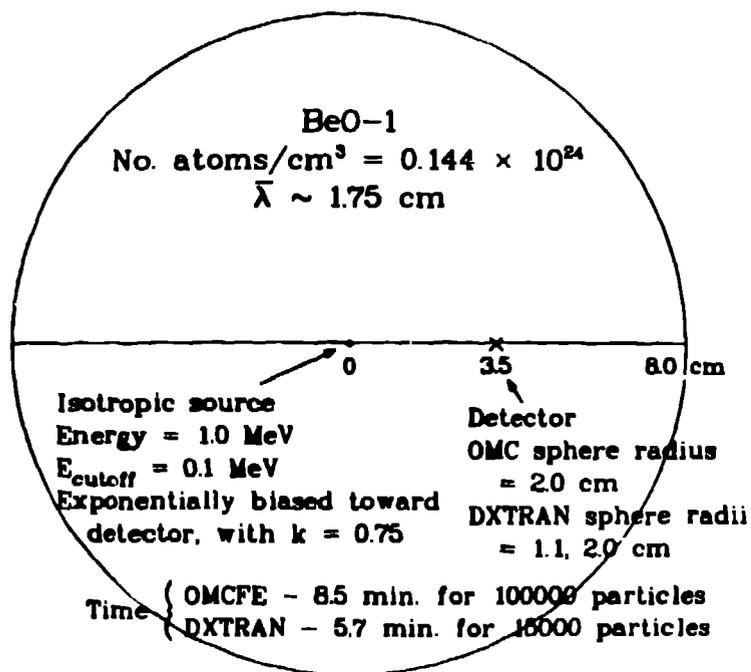


Fig. 8 Geometry for BeO-1 Problem.

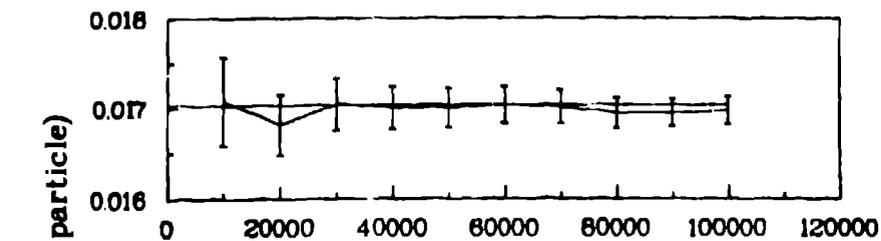


Fig. 9 Flux at a Detector in BeO-1 with the OMCFE.

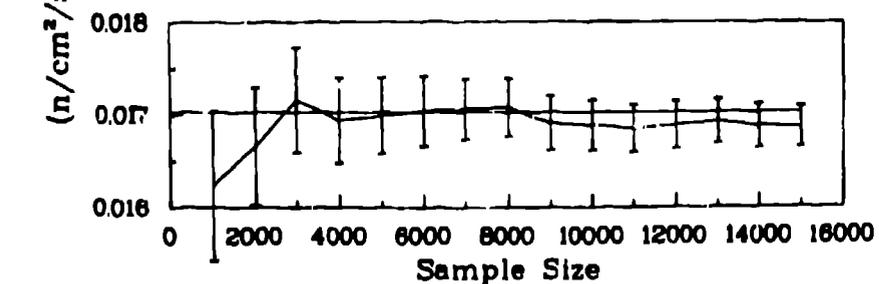


Fig. 10 Average Flux in the Vicinity of a Detector in BeO-1 Using DXTRAN.

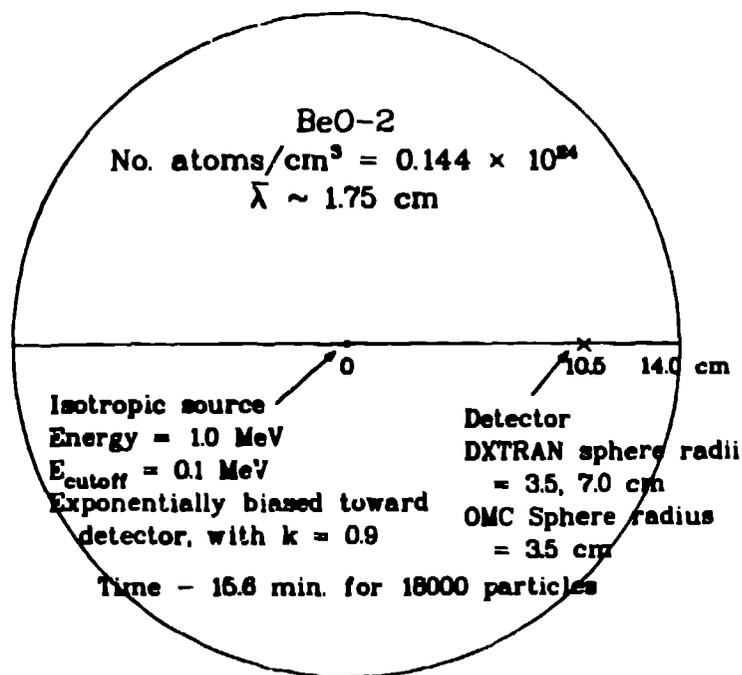


Fig. 11 Geometry for BeO-2 Problem.

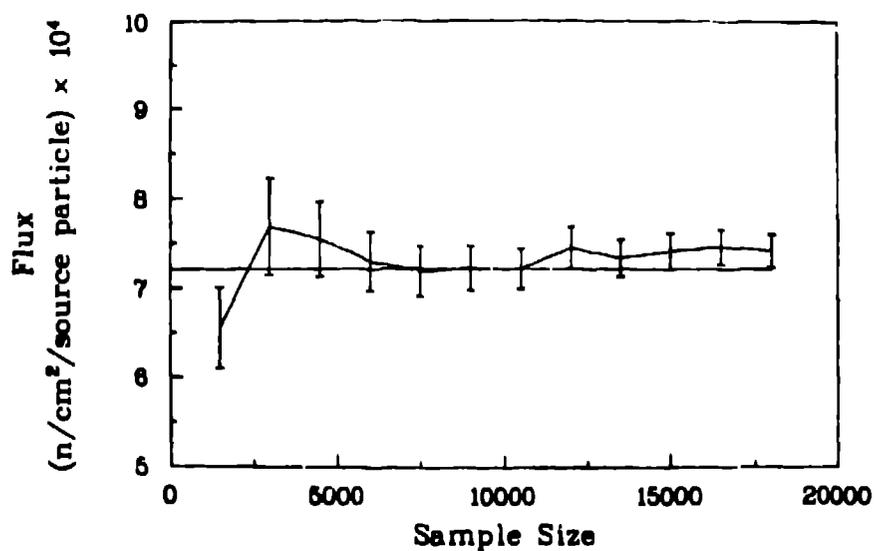


Fig. 12 Flux at a Detector in BeO-2 with the OMCFE and DXTRAN

TABLE I
Comparison of Calculated Flux with Exact Flux

		Flux (n/cm ² /source particle)			
		Exact (Surface Crossing Estimator)	OMCFE	Ave. Flux (Track Length Estimator)	Error (1 Standard Deviation)
Thermal H:	Detector 1	3.462 x 10 ⁻²	3.486 x 10 ⁻²		.056 x 10 ⁻²
	Detector 2	1.230 x 10 ⁻²	1.231 x 10 ⁻²		.032 x 10 ⁻²
Thermal CH ₂ :	Detector 1	2.086 x 10 ⁻¹	2.122 x 10 ⁻¹		.053 x 10 ⁻¹
	Detector 2	6.259 x 10 ⁻²	6.378 x 10 ⁻²		.357 x 10 ⁻²
BeO - 1:	Detector 1	1.703 x 10 ⁻²	1.697 x 10 ⁻²	1.687 x 10 ⁻²	.015 x 10 ⁻²
	Detector 1	1.703 x 10 ⁻²			.022 x 10 ⁻²
BeO - 2:	Detector 1	7.207 x 10 ⁻⁴	7.412 x 10 ⁻⁴		.185 x 10 ⁻⁴

In Problem 1 the fluxes at two detector points in thermal H are calculated using the OMCFE. Problem 2 is a similar calculation in thermal CH₂. In Problem 3, the flux is calculated at a single detector in a sphere of BeO (non-thermal) for a source of 1 MeV neutrons at the center. The flux is first obtained using the OMCFE, and this is compared with an estimate of the average flux in a sphere about the detector of 1 cm radius. The latter estimate is obtained with the help of DXTRAN. Problem 4 finds the flux at a point in a BeO sphere situated approximately 6 free paths from the source using the OMCFE, but with the aid of a large DXTRAN sphere which encloses the detector. The error-bars (one standard deviation) on the points plotted indicate the statistical accuracy of the calculation in progress, as printed out by the code. The final results are, in every case, within a few percent of the value of the "exact flux" - in fact, the agreement appears somewhat better than expected in at least one case. For example, in the BeO-1 calculation the agreement between the exact flux and that obtained from the average flux in a sphere of 1-cm radius using DXTRAN is surprisingly good. Perhaps that is fortuitous - experience does not lead one to expect it in the average problem. The amount of computing time used could have been reduced in some cases without altering the results appreciably, but in dealing with estimates of flux at a point, it pays to be reasonably cautious. Quite frequently, the calculation is sensitive to the various parameters set in a problem - the size of the imaginary sphere in the OMCFE, the source bias, etc. Some care is essential in setting up a problem and a few short runs can be invaluable in making the necessary decisions, particularly in the case of a difficult problem.

Concluding Remarks

A very important method of estimating flux at a point in a problem with axial symmetry is through the use of a ring detector. MCNP contains a ring detector option and, although we did not use it in the present calculations, it should be mentioned as one of the tools available.

While the OMCFE in MCNP can deal with neutrons thermalized according to the free gas model, there remains the task of modifying the flux estimator to be compatible with neutrons thermalized with the S(α , β) treatment. It is hoped that this defect can be rectified in the not too distant future.

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